ENDF/B-V U-233
resonance total cross section

![Graph showing the total cross section as a function of energy (MeV). The x-axis represents energy in MeV ranging from $10^{-7}$ to $10^{-6}$, and the y-axis represents cross section in barns ranging from $10^2$ to $10^6$. The line represents the total cross section with a downward trend.]
ENDF/B-V U-233
resonance total cross section
ENDF/B-V U-233
resonance total cross section
ENDF/B-V U-233
resonance absorption cross sections

Cross section (barns)

Energy (MeV)

- capture
- fission
ENDF/B-V U-233 resonance absorption cross sections

Energy (MeV) vs. Cross section (barns) for capture and fission processes.
ENDF/B-V U-233
resonance absorption cross sections

Cross section (barns)

Energy (MeV)
ENDF/B-V U-233
UR total cross section

Cross section (barns)

Energy (MeV)
ENDF/B-V U-233
UR elastic cross section

Energy (MeV)

Cross section (barns)

- Inf. Dil.
- 100 b
- 1 b

Effusion Dil.
ENDF/B-V U-233
UR capture cross section

Energy (MeV)

Cross section (barns)

Inf. Dil.
100 b
1 b
ENDF/B-V U-233 Heating

Energy (MeV) vs. Heating (MeV/reaction)
ENDF/B-V U-233
Damage

Energy (MeV) vs. Damage (MeV-barns)
ENDF/B-V U-233
Non-threshold reactions

Energy (MeV)

Cross section (barns)

- fission
- (n,gma)
ENDF/B-V U-233
Principal cross sections

Energy (MeV)

Cross section (barns)

- total
- absorption
- elastic

Energy (MeV)

Cross section (barns)
ENDF/B-V U-233 Damage

Damage

Energy (MeV) vs. Damage (MeV-barns)
ENDF/B-V U-233
Non-threshold reactions

Energy (MeV)

Cross section (barns)

- fission
- (n,gma)

Energy (MeV)
ENDF/B-V U-233 angular distribution for elastic
ENDF/B-V U-233
Delayed neutron spectra

Energy (MeV) vs Probability

- Group 1: 0.0860 decay/shake 1.258E-10
- Group 2: 0.2740 decay/shake 3.342E-10
- Group 3: 0.2270 decay/shake 1.310E-09
- Group 4: 0.3170 decay/shake 3.030E-09
- Group 5: 0.0730 decay/shake 1.270E-08
- Group 6: 0.0230 decay/shake 3.140E-08