Fission Cross Sections

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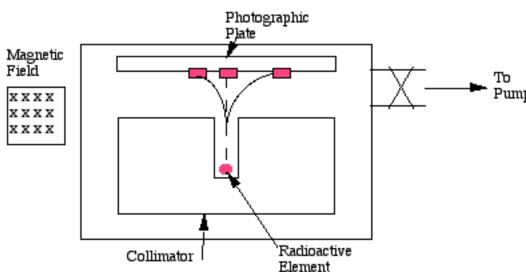
Outline

- Introduction events leading to experimental evidence of nuclear fission
- Fission Phenomenology and Observables
- Fission Cross Sections Using Neutrons
- Fission Cross Sections from Charged Particle Measurements
- The Future of Fission Cross Section Measurements and Possible Improvements

1896: Becquerel accidentally discovers radioactivity from uranium ore and discovers that deflection in a magnetic field led to positive, negative, and neutral classes of radioactivity.

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has a bound 'electron-proton pairs' to explain the disparity between element's atomic mass

and number. Does the inertia of a body depend upon its energy-content?

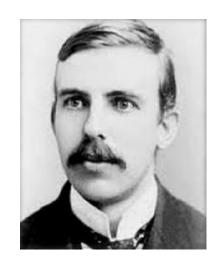


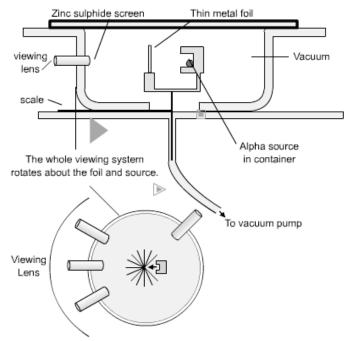
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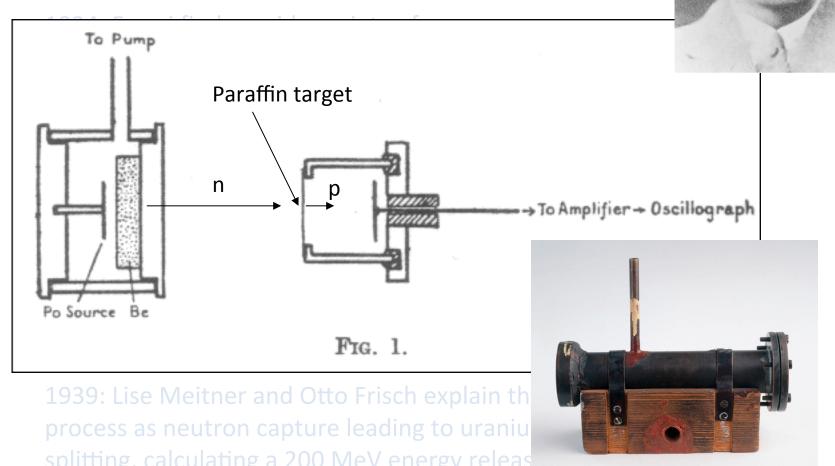
1906: Rutherford discovers α particles and develops scattering apparatus – results lead him to conclude that atoms have a 'nucleus' of positive charge.

1920: Rutherford postulates that the nucleus has bound 'electron-proton pairs' to explain the disparity between element's atomic mass and number.





1932: Chadwick bombards Beryllium with α particles: proposes that the 'electron-proton pairs' were really a new particle – the neutron.



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1934: Fermi finds a wide variety of radionuclides from irradiating uranium using a Po-Be source including lighter elements

1934: Ida Noddack publishes a paper discussing Fermi's radiochemistry and suggests 'It is conceivable that the nucleus breaks up into several large fragments ..."

1938: Otto Hahn and Fritz Strassmann irradiate uranium and show that the lighter elements seen by Fermi were about ½ the mass of the uranium, demonstrating fission

1939: Lise Meitner and Otto Frisch explain the process as neutron capture leading to uranium splitting, calculating a 200 MeV energy release.







several large fragments ...

1938: Otto Hahn and Fritz Strassmann irradiate uranium with neutrons and identify Barium, showing that the lighter elements earlier seen by Fermi were about 1/2 the mass of the uranium, demonstrating fission.



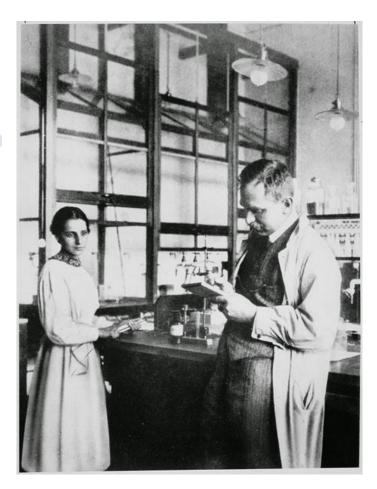


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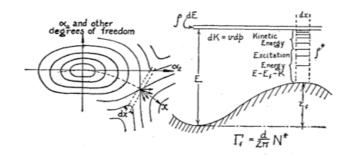


Fig. 3. The potential energy associated with any arbitrary deformation of the nuclear form may be plotted as a function of the parameters which specify the deformation, thus giving a contour surface which is represented schematically in the left-hand portion of the figure. The pass or saddle point corresponds to the critical deformation of unstable equilibrium. To the extent to which we may use classical terms, the course of the fission process may be symbolized by a ball lying in the hollow at the origin of coordinates (spherical form) which receives an impulse (neutron capture) which sets it to executing a complicated Lissajous figure of oscillation about equilibrium. If its energy is sufficient, it will in the course of time happen to move in the proper direction to pass over the saddle point (after which fission will occur), unless it loses its energy (radiation or neutron re-emission). At the right is a cross section taken through the fission barrier, illustrating the calculation in the text of the probability per unit time of fission occurring.

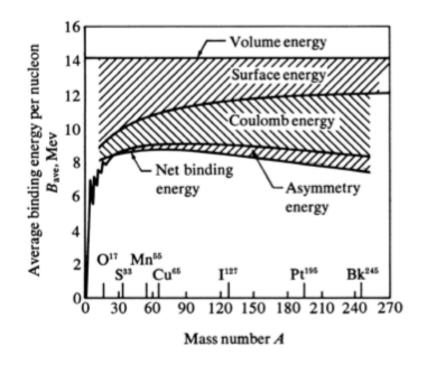
Fission Phenomenology and Observables Liquid Drop Model

The liquid drop model developed by Gamow and formulated by Weizsäcker in 1935 treats the nucleus as a uniformly charged incompressible liquid drop with the binding energy of the nucleus given as:

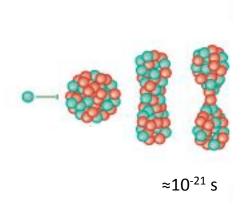
$$B = a_v A - a_s A + a_c Z^2 / A^{1/3} - a_A (A-Z)^2 / A \pm \delta / A^{1/2}$$

Volume Surface Coulomb Isospin Pairing

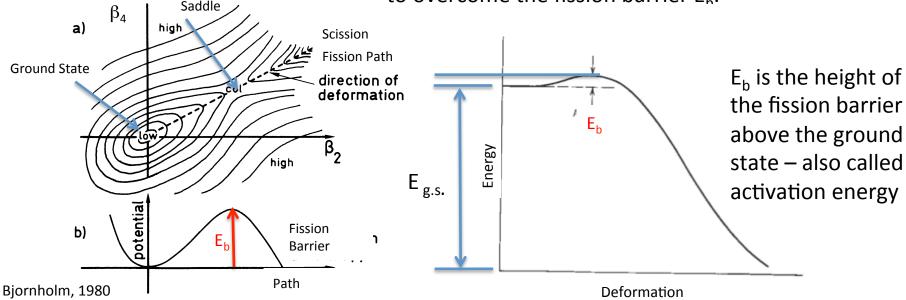
In 1939, Bohr and Wheeler applied this model to explain fission. For a liquid drop, of constant volume, deformation information is all that matters, so only the Coulomb and surface energy terms are important.



Fission Phenomenology and Observables A Simple Picture of a Neutron-Induced Fission Event

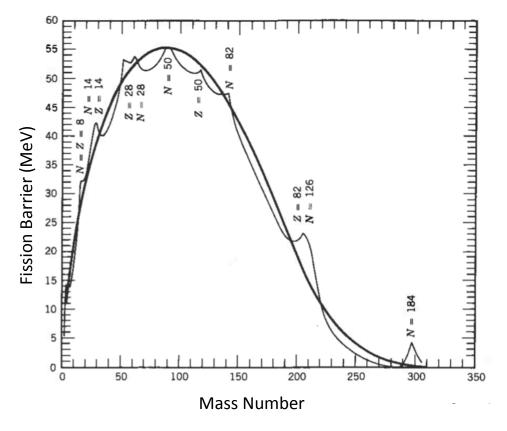


With the deformation energy in the liquid drop as the difference between the surface and Coulomb terms, Bohr and Wheeler described the nuclear deformation by a Legendre expansion about the nuclear radius $R(\theta)$ with coefficients β_n . In the $(\beta_2 - \beta_4)$ plane they found a saddle point. Beyond that the minimum energy path slopes downward allowing the nucleus to break apart. The fissioning nucleus just has to have enough energy to overcome the fission barrier E_h .

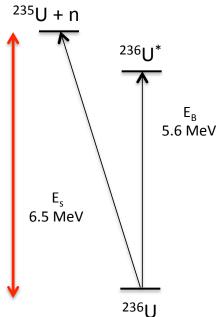


Fission Barrier Heights and Energy Level Diagrams for ²³⁵U and ²³⁸U

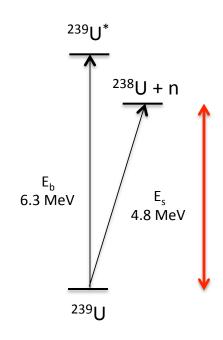
Formation of ²³⁶U* and ²³⁹U* can lead to fission.



Fission Barrier Height variation with mass number calculated from LDM for the most stable isotope at each mass number. The light curve includes shell structure. Myers Nucl Phys, 81, 1 (1966)

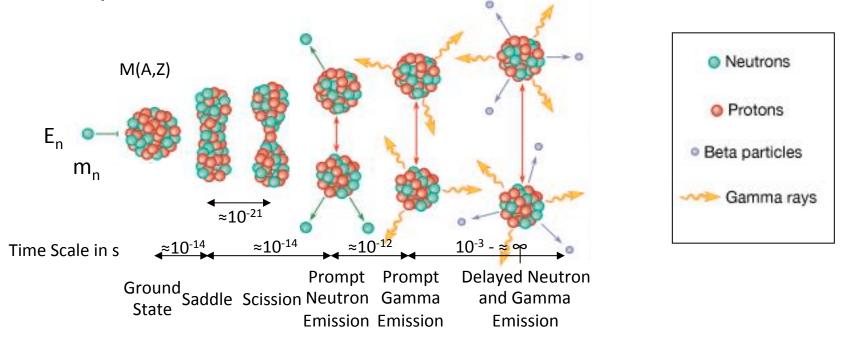


energy than the lowest fissionable state, so it can fission with a zero-energy neutron



²³⁸U + n has a lower energy than the lowest fissionable state, so it requires additional 1.5 MeV of kinetic energy to fission

Fission Phenomenology and Observables A Simple Picture of a Neutron-Induced Fission Event



Total available energy: $Q = M(Z,A) - M_L(Z_L,A_L) - M_H(Z_H,A_H) + m(n) + E_n$

The fission fragments are generally left in an excited state and the energy is distributed between the kinetic energy and excitation energy, denoted as E_{TKE} and E_{TXE} .

Where E_{TXE} is the total fragment internal energy resulting from the fragments changing to their equilibrium deformation after scission.

Fission has a complex set of observables, measurements of each of which provide information that helps us understand the overall process

- Energy Release
- Fission cross sections and probabilities for different incident particles and energies
- Multiplicity, Energy, and Angle Distributions of Neutrons and Gamma Rays from Fission
- Fragment Kinetic Energy, Mass, and Charge Yields and Angular Distributions
- Fragment Delayed Neutrons
- Fragment Beta and Gamma Decay Processes
- Spontaneous Fission
- Spin and parity distributions of fission resonances
- Parity violation in fission resonances
- Fission Time Scales
- ...

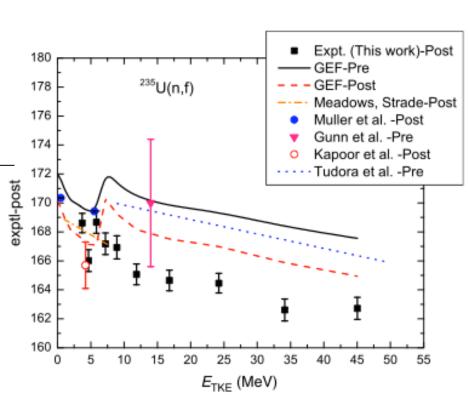
Fission can occur when a heavy compound nucleus has an energy that exceeds the fission barrier, B_f . That can happen when struck by neutrons, charged particles, or gamma rays. It can also occur in ground-state nuclei through quantum-mechanical tunneling, resulting in 'spontaneous' fission.

Fission Phenomenology and Observables Fission Energy Release

²³⁵U thermal neutron fission

Approximate Prompt Energy Release	MeV	
Fission Fragments	168	
Fission Neutrons	5	
Prompt γ emission	7	
γ emission from fission fragments	7	
eta emission from fission fragments	8	
u from fission products	12	
Total Energy per Fission	207	*

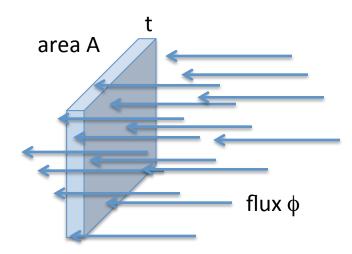
When resulting from more energetic particles the energy released by fission fragments has an energy dependence, shown here for post neutron emission.



Excitation energy dependence of the total kinetic energy release in $^{235}\text{U}(\text{n,f})$ Yanez, 2014

See also Madland, 2006

Cross Sections



 $I = \phi \cdot A$ (number of particles/s)

The interaction rate dN/dt for incident particles on a surface of area A and thickness t depends on the intensity and the number of nuclei exposed to the incident particles:

The intensity I (number of particles/s) is defined in terms of a flux ϕ , or number of particles/cm²/s striking an area A

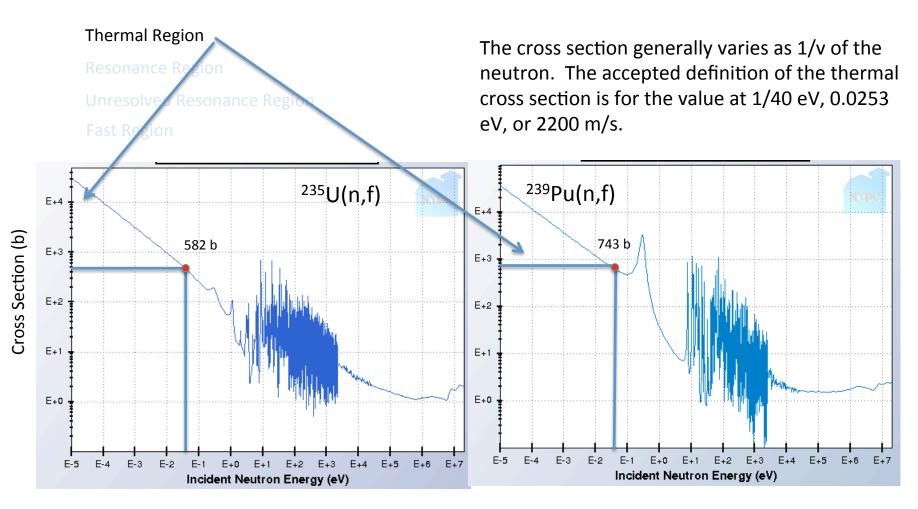
For N_t target nuclei with number density ρ (#/cm³) exposed to the incident particles, then

$$N_t = \rho \cdot A \cdot t$$

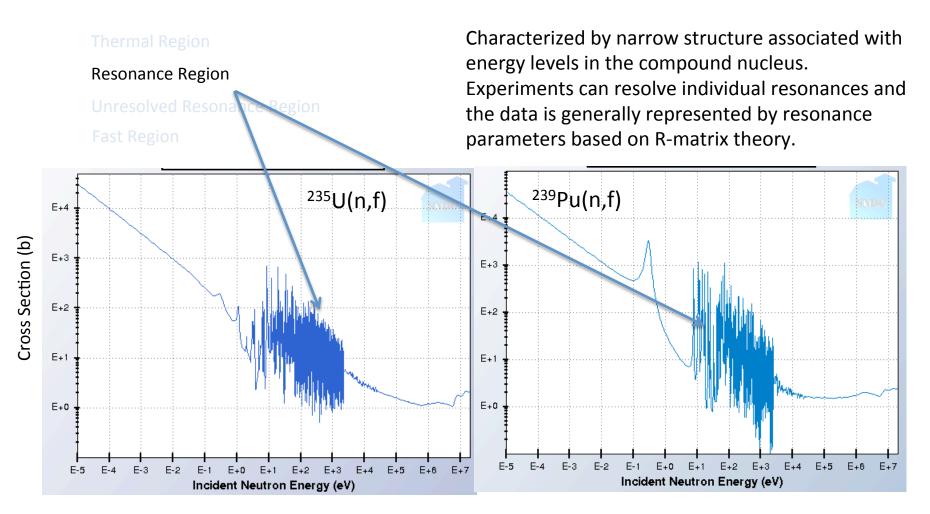
The interaction rate N will be proportional to the number of incident particles (I) and number of atoms (n) $dN/dt \propto \varphi \cdot N_t \\ dN/dt \propto \cdot \varphi \cdot \rho \cdot A \cdot t$

 $dN/dt = \sigma \cdot \rho \cdot A \cdot t \cdot \phi$

 σ = event rate per atom/incident flux with units of area It is the probability that that a particle will interact with a nucleus, or the effective cross sectional area of the nucleus. Units are barns or 10^{-24} cm² Note: $dN/dt = \sigma \cdot N_{t} \cdot \phi$



Neutron Induced Fission Cross Sections have a complex energy dependence



Neutron Induced Fission Cross Sections have a complex energy dependence

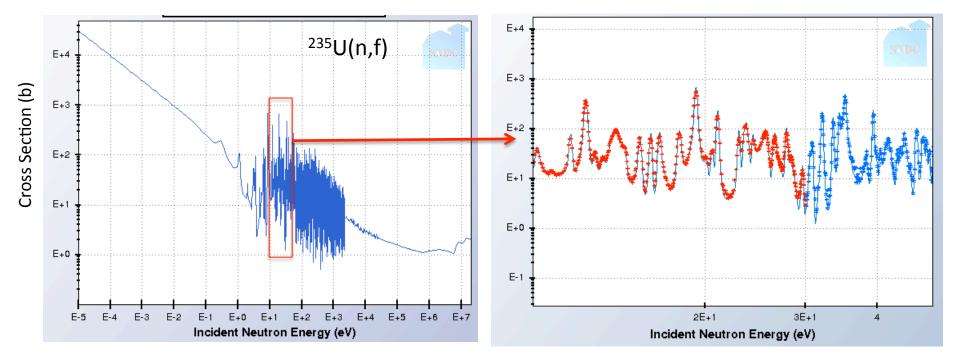
Thermal Region

Resonance Region

Unresolved Resonance Region

Fast Region

Expansion of resonance region for ²³⁵U showing ENDF/B-VII.1 evaluation(solid line) and one set of experimental data from Wagemans (1979)



Neutron Induced Fission Cross Sections have a complex energy dependence

In this region the cross section still fluctuates, but levels in the compound nucleus are too closely spaced to be resolved. The cross section is **Unresolved Resonance Region** approximated by average resonance parameters using statistical and level density models. ²³⁵U(n,f) ²³⁹Pu(n,f) E+4 Cross Section (b) E+3 E+2 E+ E+1 E+0 E+0 E+2 E-3 E+5 E-4 E-3 E+0 E+2 E+5 Incident Neutron Energy (eV) Incident Neutron Energy (eV)

Neutron Induced Fission Cross Sections have a complex energy dependence

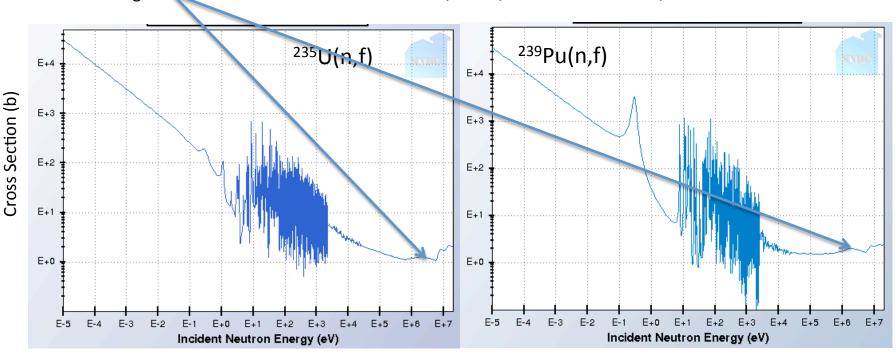
Thermal Region

Resonance Region

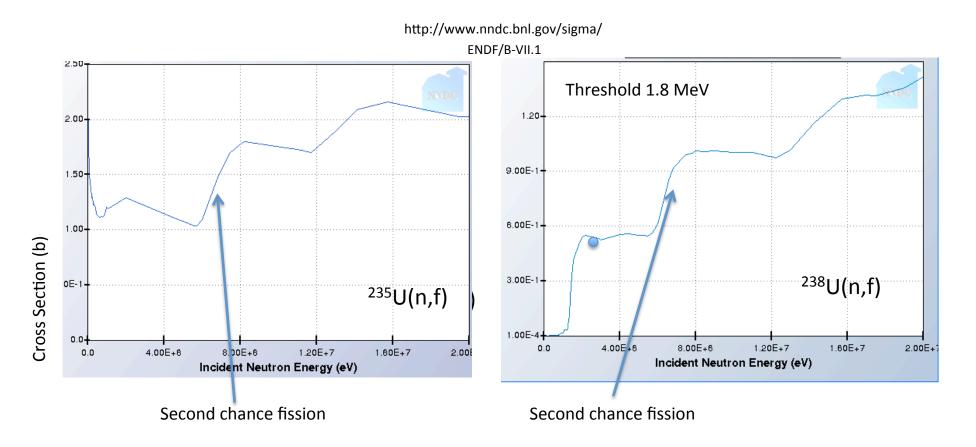
Unresolved Resonance Region

Fast Region

Levels overlap so strongly that there are no longer fluctuations, cross sections are smooth. They are represented by statistical, intra-nuclear cascade, pre-equilibrium, and evaporation models.



Fission Phenomenology and Observables Even-even nuclei have fission thresholds

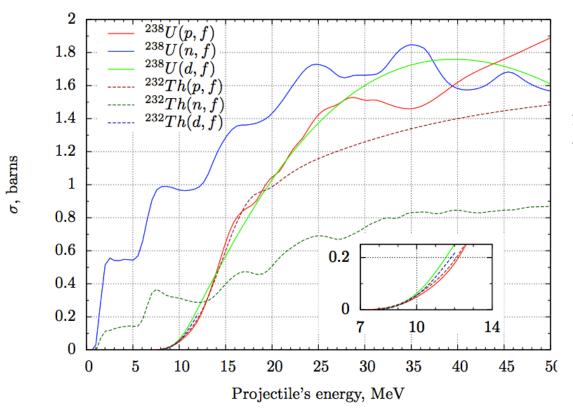


Fast neutron induced fission is generally smooth with regular steps as the incident neutron energy increases enough to allow the compound nucleus to emit one neutron and still have enough energy to fission (n,n'f). This is 'second chance' fission. At higher incident energies, 'third chance' fission becomes possible.

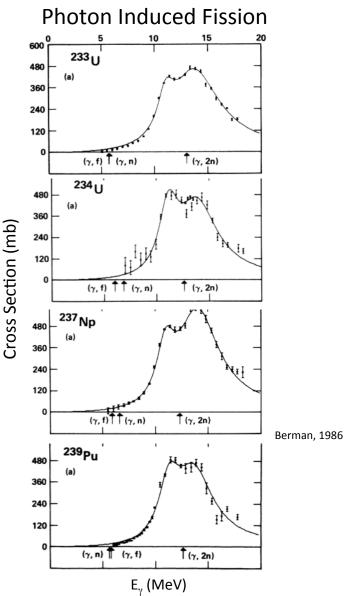
Fission induced by other particles than neutrons provide complimentary

information

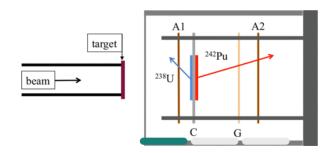
Cross Sections from proton, neutron, and deuteron-induced fission induced fission



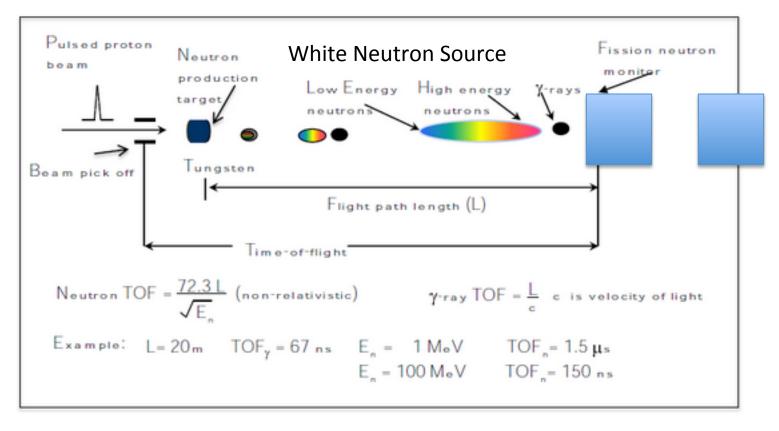
"Nuclear fission studies with the IGISOL method and JYFLTRAP", D. Gorelov Dissertation (2015)



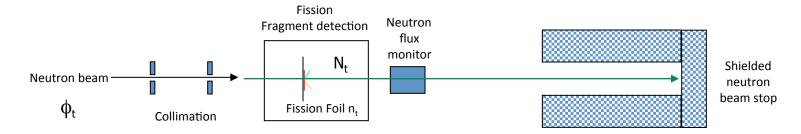
Typical Fission Neutron Cross Section Arrangements



Monoenergetic Neutron Source



Fission Cross Cross Section Measurement Example



$$\begin{split} & dN_t/dt = & \varphi_t \cdot \sigma_f n_t \\ & N_t = \int \! \varphi_t \cdot \sigma_f n_t dt \\ & N_t = & \sigma_f n_t \cdot \Phi_t \end{split}$$

 $\sigma_{\text{f}} = N_{\text{t}}(\text{E})/(\Phi_{\text{t}}(\text{E})\cdot n_{\text{t}}) = N_{\text{t}}/(\Phi_{\text{t}}(\text{E})\cdot n_{\text{t}}) \text{ where } N_{\text{t}}(\text{E}) \text{ is the number of detected events, } \Phi_{\text{t}}(\text{E}) \text{ is the neutron fluence to which the fission foil was exposed and } n_{\text{t}} \text{ is the atom density of the sample}$

$$\mathsf{N}_\mathsf{t}(\mathsf{E}) = \omega_\mathsf{t}(\mathsf{E}) \cdot [\epsilon_\mathsf{t}(\mathsf{E}) \cdot \Phi_\mathsf{t}(\mathsf{E}) \cdot \sigma_\mathsf{f}(\mathsf{E}) \cdot \mathsf{N}_\mathsf{t}(\mathsf{E}) + \mathsf{N}_\mathsf{b}(\mathsf{E})]$$

 $N_{bt}(E_n)$ represents a background from detected alpha particles from radioactive decay or from events not associated with the primary beam. Inefficiencies in fragment detection $(\varepsilon_t(E))$ and for computer and electronics livetime $(\omega_t(E_n))$ are also included.

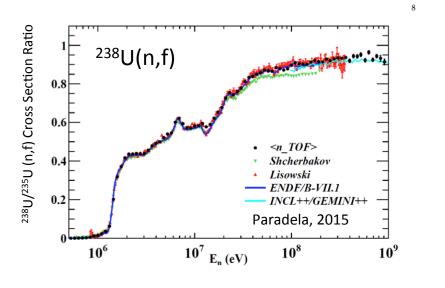
$$\sigma_{f}(E) = [1/\omega_{t}(E) \cdot N_{t}(E) - N_{bt}(E)] / [\varepsilon_{t}(E) \cdot n_{t} \cdot \Phi_{t}(E)]$$

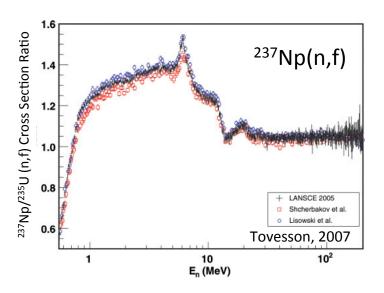
Except for determination of the neutron fluence, each of the corrections are reasonably straightforward. For count rates generally found in fission experiments with modern electronics livetime is a constant.

For many experiments, performing measurements of fission cross section ratios to ²³⁵U instead of determining absolute fission data makes it possible to rely on a single standard cross section, ²³⁵U(n,f) and avoid making precise measurements of the incident neutron fluence. To the extent that the fluence, livetime and backgrounds are similar, cancellations occur.

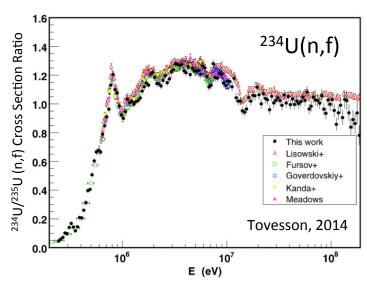
$$\sigma_{\rm f}(E)/\sigma_{235}(E) = n_{235}/n_{\rm t} \cdot \epsilon_{235}(E)/\epsilon_{\rm t}(E) \cdot \Phi_{235}(E)/\Phi(E) \cdot [N_{\rm t}(E)/\omega_{\rm t}(E) - N_{\rm bt}(E)]/[N_{235}(E)/\omega_{235}(E) - N_{\rm b235}]$$

Fission Cross Section Ratio Measurements



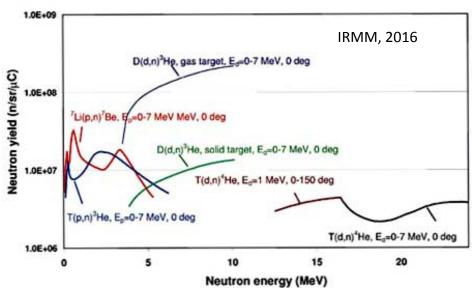


- Ratio measurements offer the best opportunity for low systematic error.
- For ²³⁸U/²³⁵U all recent ratio data are in good agreement
- There are still differences of order 3
 5% for many other ratio data



Neutron Sources – mono-energetic or quasi mono-energetic

Mono-energetic or quasi mono-energetic neutron sources are generally made through light-ion reactions such as T(d,n), T(p,n), D(d,n), ⁷Li(p,n) and ⁹Be(p,n) and are generally good sources for fast neutron measurements. Inverse kinematics such as H (⁷Li,n) can be used.



The most common accelerators used for mono-energetic neutron production are van de Graaff machines, examples used for neutron physics are:

Facility Triangle Universities Nuclear Laboratory (TUNL) at Duke University	Particles H,D	Max Energy 20 MeV	Pulse Width Ty 1 ns	p. Beam Current 1 – 5 μΑ
Edwards Accelerator Laboratory at Ohio University	H, D, ³ He, ⁴ He	9 MeV	1 ns	1-3 μΑ
Institute for Reference Materials and Measurements (IRMM), Geel Belgium	H,D	14 MeV	2 ns	1-3 μΑ

Neutron Sources – White Neutron Sources and Reactors

Multi-spectral neutron sources generally use energetic beams of protons or electrons striking neutron production targets. Neutron energies are then determined using time-of-flight techniques over relatively long flight paths to allow studies as a function of neutron energy..

Examples of neutron sources available to users for fission research:

Facility LANSCE-WNR	Particles 800-MeV p	n Energy n 0.1- 600 MeV	Pulse Width 0.5 ns	Flight Paths 6-25 m
LANSCE-Lujan	800-MeV p	Thermal – 0.1 MeV	125 ns	6-25 m
GELINA	100 MeV e ⁻	Subthermal – 20 Me	V 1 ns	10 – 400 m
RPI	100 MeV e ⁻	0.01 eV to 1 keV	6 ns	25 m
nTOF (CERN)	20 GeV p	meV – GeV	6 ns	185 m

Facilities such as the Lohengrin spectrometer at the ILL reactor allow unique spectroscopic measurements of mass and isotopic yields for fission products.

Types of Fission Fragment Detectors

Proportional Counters - most common technique Parallel plate chamber for fragment detection Gridded to improve S/N and for angular information

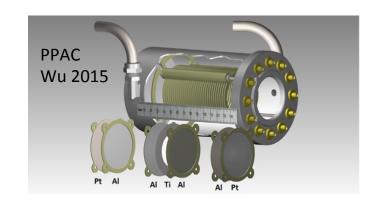
Parallel-plate avalanche counter (PPAC)
Low energy resolution
Very fast response (sub ns)

Gas scintillation chambers

Noble gas mixtures (90% ³He, 10% Xe) at high pressure Tovesson, 2007 UV light requires a wavelength shifter

Solid-state silicon detectors

Good pulse height information and timing
Good energy resolution
Small solid angle



Beam

Direction

Retaining ring

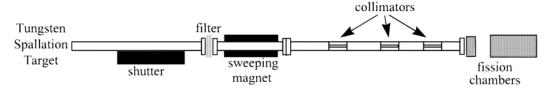
Windows

Teflon spacer

Time Projection Chamber

Neutron Beam and Flight Path Considerations

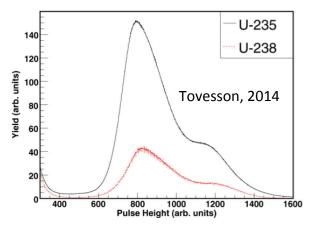
- Time marker for pulse generation t₀
- Adequate Collimation -Illumination of fission mass uniformly without having neutrons striking structure
- Removal of potential neutron beam contaminants
 - Charged particles, gamma rays Frame overlap for pulsed beam measurements
- Incident neutron beam quality
 Uniformity, extent at experiment
- Adequate shielding neutron beam dump Adjacent flight paths (sky shine)
- Appropriate neutron flight path length



Neutron Flight Path at WNR

Fission Fragment Detection

- Accurate determination of fission foil masses
- Adequate electronics for pulse height and timing and gain stability
- Adequate fission-fragment to decay alpha pulse height separation
- Cooling if needed to reduce Doppler broadening in resonance region

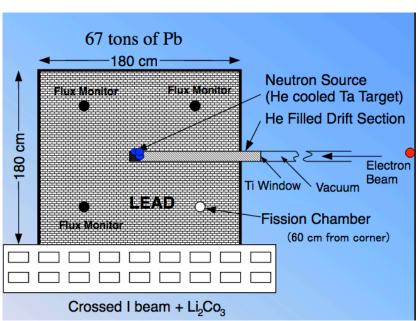


Parallel Plate ion Ionization Chamber Fission Fragment Response

Lead Slowing Down Spectrometer (LSDS)

Neutrons are produced by a multi-spectral pulsed neutron source inside a large block of lead. Useful for measuring fission cross sections of quantities of actinides to small for other techniques. This technique is especially useful for samples and highly radioactive materials over the energy range 0.1 eV to 100 keV

LSDS sources are possible at RPI, j-PARC, and WNR (Target-2):

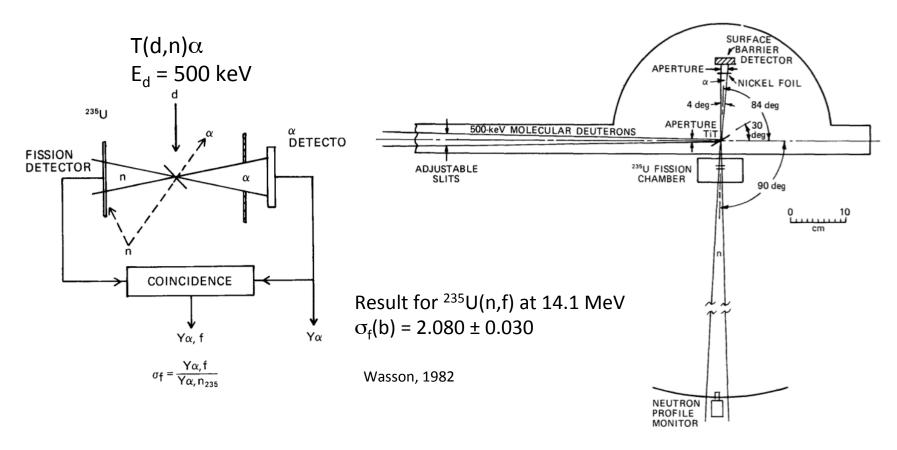


LSDS at RPI Linac

- Ta target at center produces neutrons
- Neutrons slow down by elastic and inelastic scattering
- Neutrons produce a broad highintensity, 30-40% resolution pulse
- Neutrons traverse the detector multiple times so the flux is 10³-10⁴ higher that that on a flight path of equivalent time-of-flight resolution (5.6 m)

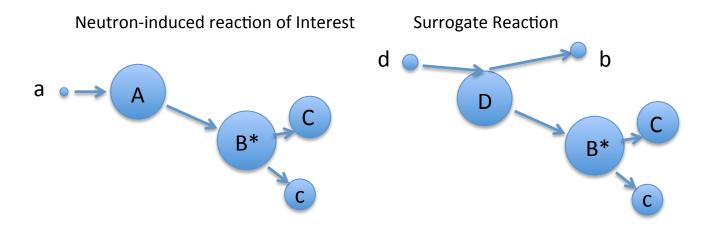
Absolute fission cross section data require an accurate neutron flux measurement

- Use of reference cross section standards H(n,p), ⁶Li (n,α) (www-nds.iaea.org/standards/)
- Black detector
- Activation Foils
- Associated particle technique (monoenergetic sources)



Surrogate Cross Section Techniques Using Charged Particle Reactions

- Many minor isotopes important to fission technology or to theoretical understanding of fission are hard to obtain in sufficient quantity or too short half-lives for a direct neutron measurement.
- A light-ion charged particle reaction is used to form the same compound system B* as in neutron-induced fission.

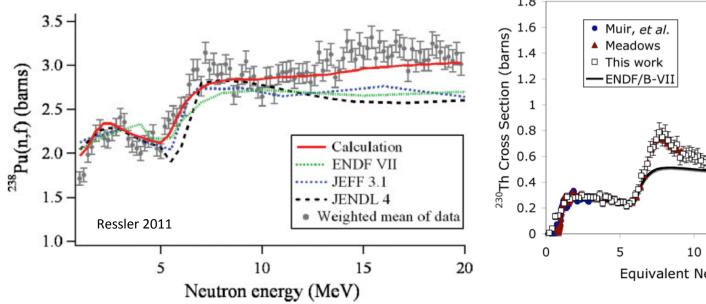


• The decay of B* is detected in coincidence with the reaction product b. The energy of b fixes E*, the excitation energy of B*.

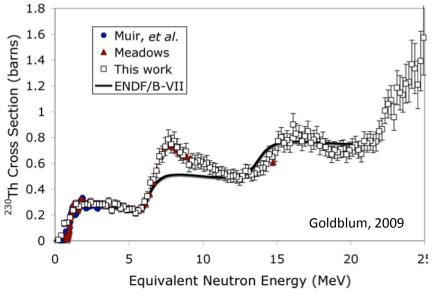
Surrogate Cross Section Techniques Using Charged Particle Reactions

- Several techniques have been developed to extract estimates of the neutron induced cross section from the measured fission probability. In the absolute Surrogate Method, Hauser-Feshbach theory or optical model potentials are used to describe the compound reaction formation for neutrons on target A.
- The fission probability is then $P_f(E^*) = N_{cf}/[N_t \cdot \epsilon(E^*)]$. N_{cf} is the number of recorded coincident particle b-fission events, and $\epsilon(E^*)$ is the fission detector efficiency.
- The neutron-induced fission cross section is then computed using the expression: $\sigma_{nf}(E_n) \approx P_f(E^* = S_n + A/(A+1) \cdot En) \cdot \sigma^{CN}_n(E_n)$.
- $\sigma^{CN}_{n}(E_{n})$ is the calculated compound nuclear formation cross section for neutrons incidend on target A.
- In the Surrogate Ratio Method, the same surrogate reaction is carried out on two different targets in order to measure a ratio of compound decay probabilities.

Surrogate Cross Section Techniques Using Charged Particle Reactions - Examples



Surrogate Ratio Method Determination of 238 Pu(n,f) cross section from 235 U(α , α f) and 236 U(α , α f) reactions.

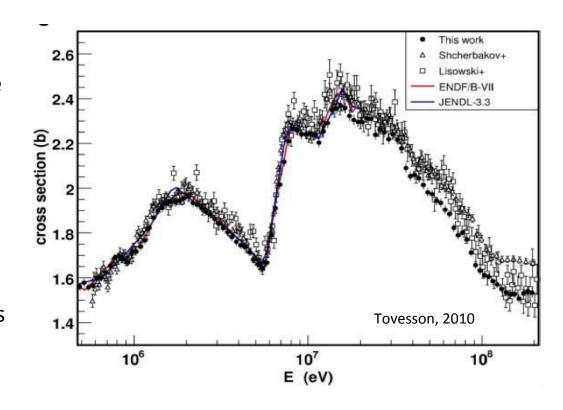


Surrogate Ratio Method Determination of 230 Th(n,f) cross section from 232 Th(3 He, α f) and 234 U(n,f) reactions. Ressler 2011

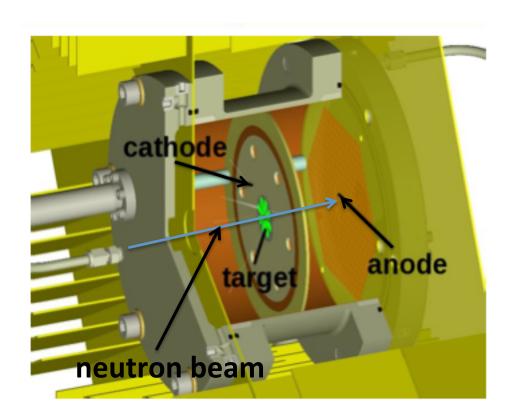
Important fast neutron fission cross sections such as those for ²³⁹Pu(n,f) have a 3-6% spread in values for experiments claiming 1-6% uncertainties. This potentially indicates unexplained systematic uncertainties.

Applications for both national defense and future fast reactor fuel development require more confidence in the actual value.

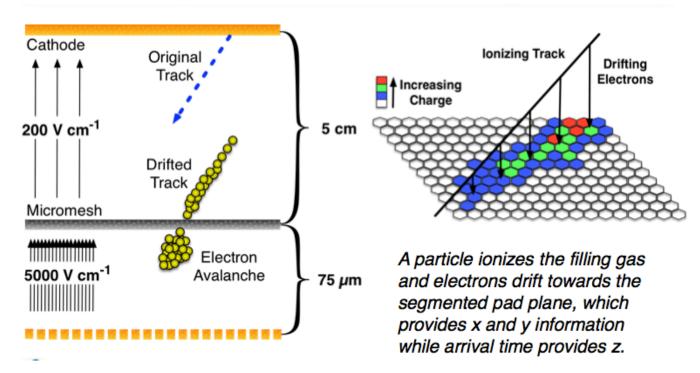
By incorporating particle tracking techniques developed for high-energy particle detectors, the goal of the TPC collaboration (NIFFTE) is to reduce systematic uncertainties in a credible way to 1% or less for important fission cross sections.



- The TPF uses a two-volume ionization chamber with a segmented anode plane allowing detection of the track of an ionizing particle
- Drift of the ionized gas to a segmented anode gives an x-y signature of the track, and arrival time provides z.
- The ionization profile of the particle track allows particle identification.
- The geometry of the TPC allows nearly 2π coverage of particles emitted from the target.

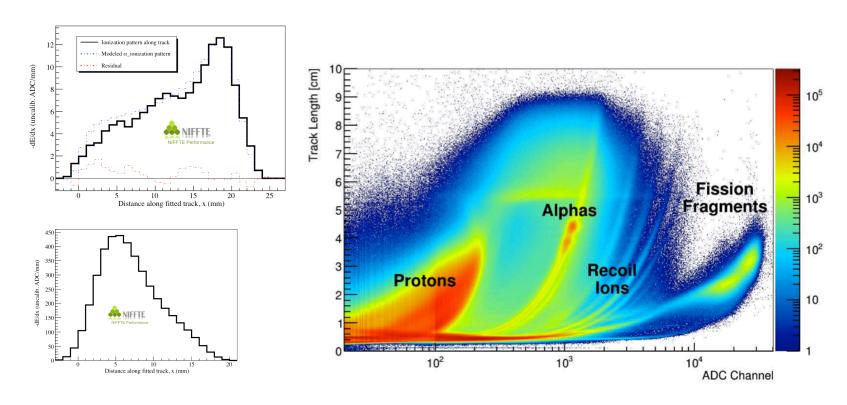


- The TPC uses a two-part system for ionization tracking. Initially the track drifts in a 500 V/cm region
- After drifting 5 cm, the track passes through a micromesh and enters a region with 5000 V/cm, creating an electron avalanche amplification



The amplified signal is detected on the anode plane with 5952 hexagonal pads.

Time Projection Chamber

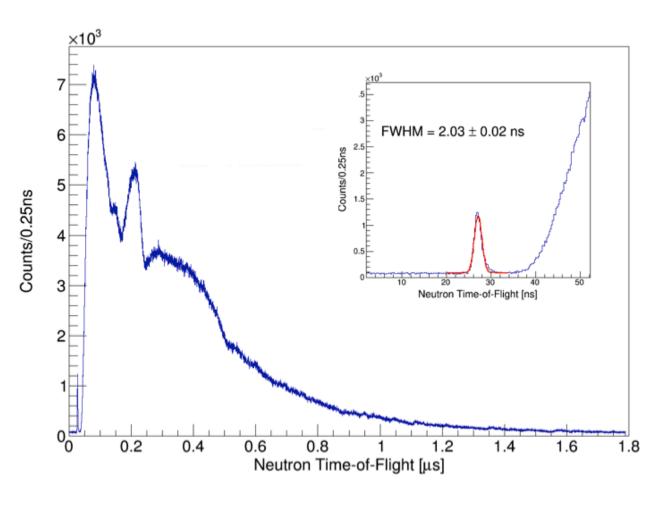


Ionization profiles of decay α and fission products allow particle identification and 2D analysis provides clean separation between the fission fragments from other reactions such as with the argon-isobutane multiplying gas.

- Neutron energy information from time-of-flight
- Energy resolution determined by 2 ns FWHM of target gamma ray pulse

This gives:

E _n (MeV)	Δ E(keV)	%
1	6	0.6
5	70	1.4
10	198	2.0
15	365	2.4
20	565	2.8



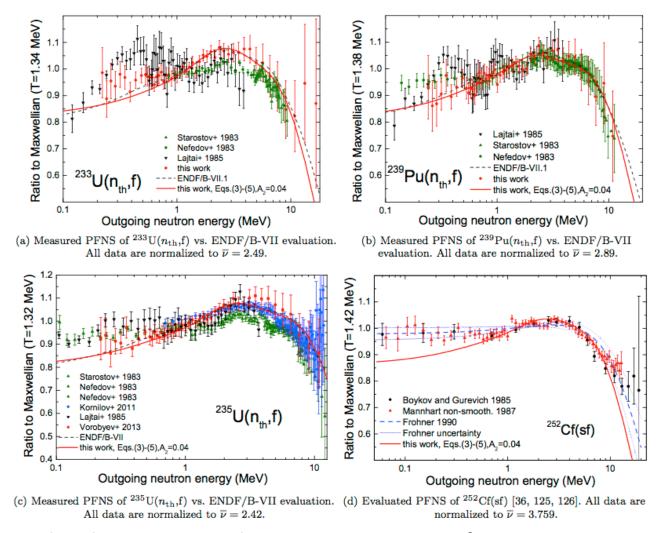
- The TPC has the potential to measure high-precision neutron-induced actinide cross sections with an unsurpassed ability to understand all features of the measurement in-situ in order to produce high-accuracy, precision results.
 - Particle identification
 - Target thickness and uniformity
 - Target isotopic content
 - Fragment emission angular distributions
- The TPC has other potential uses:
 - Fission cross sections for binary and ternary fission
 - Fission cross section ratios
 - Neutron-induced reaction cross sections
 - Improved data for cross section standards such as H(n,p), ⁶Li (n,α) , ...

The Future of Fission Cross section Measurements and Possible Improvements

- Improved confidence in our current knowledge of neutron-induced fission cross sections, including sufficient information to evaluate covariance data over all energy ranges, is needed for national security applications.
- Development of new fission applications for technology will be a driver for future fission cross section measurements
 - Generation IV reactors and research on actinide burning systems require better accuracy in both data and higher confidence in evaluations. Some measurements may be possible with a LSDS for very small or highly radioactive samples.
- Continued use of the surrogate technique for actinide fission where samples are unsuitable for direct measurement
- The Time Projection Chamber is a significant step forward in our ability to make accurate and precise measurements because of the increased ability to understand the target and detection process in detail.
 - Once fully developed, expanding the energy range of the TPC to lower energies would allow measurements in an energy range with significant national security and civilian energy importance.

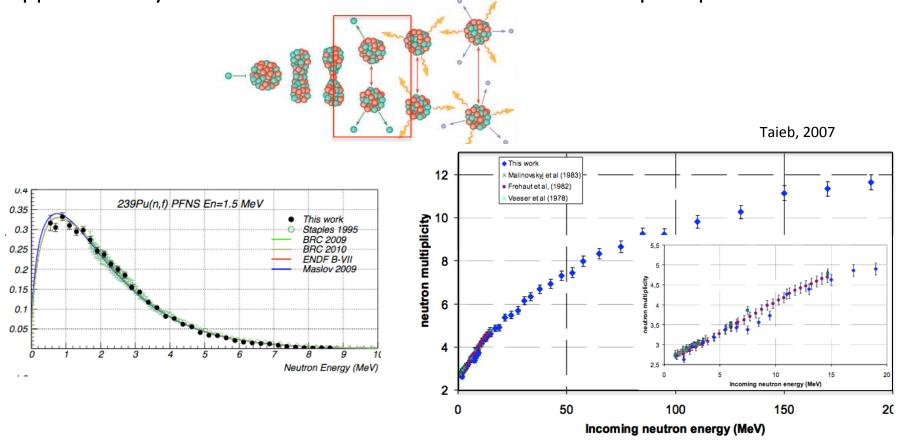
Backup Material

Prompt Neutron Fission Spectra



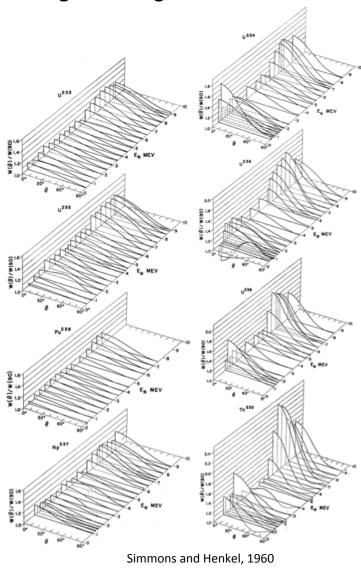
Measured and parameterized PFNS using equations from Capote, 2016.

Approximately 99% of the neutrons released in fission are prompt

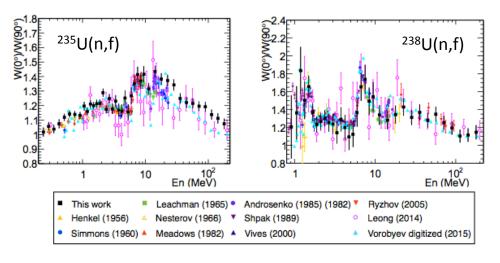


The laboratory neutron energy distribution at each energy is approximated by a Maxwellian Spectrum. The average number of prompt neutrons is \approx 2.5, with an average energy of \approx 2 MeV. This neglects several important physical effects and modified version presented by Capote, 2016, provides a better description.

Fragment Angular Distributions

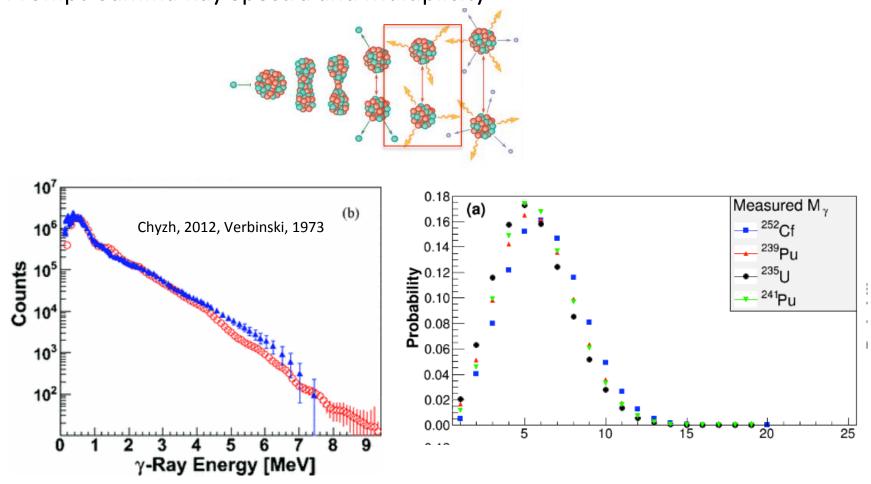


Fragment distributions are often parametrized by the anisotropy W(0°)/W(90°). The angular distributions are not simple functions of incident neutron energy as shown by recent data From CERN.



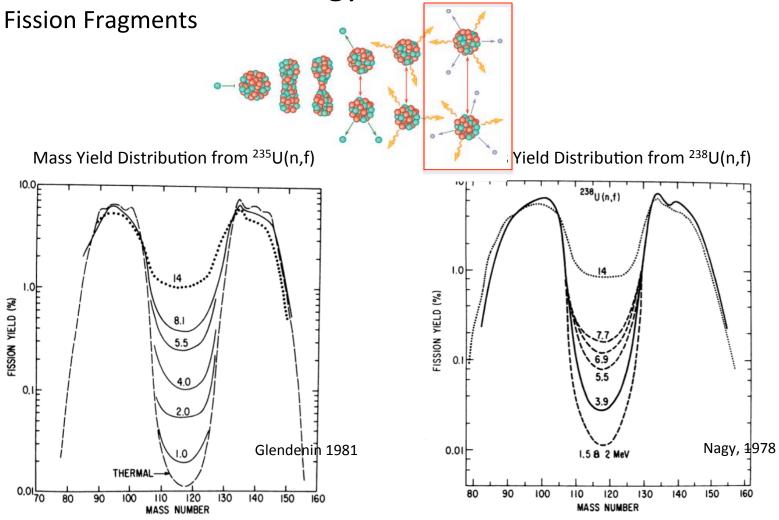
E. Leal-Cidoncha, 2016

Prompt Gamma Ray Spectra and Multiplicity



Total ²³⁵U fission prompt gamma ray emission distribution for neutrons 0.025 eV to 100 keV

Total prompt fission gamma ray multiplicity from 0.025 eV to 100 keV



Mass yield for symmetric fission increases with energy, rising by 3 orders of magnitude from threshold to 14 MeV in ²³⁵U(n,f) and 2 orders of magnitude in ²³⁸U(n,f)